



Non radioactive

waste (NR)

Tape

or 5 Shield

Variable resource
expected to be recycled

Evaluation of radioactive concentration produced in electric equipments and materials on the decommissioning of nuclear power plants

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1. Introduction

- ✓ The aging nuclear power plants will increase in the coming decades, and most of them are expected to be decommissioning
- ✓ If the part of waste generated in the decommissioning could be treated as nonradioactive wastes, the amount of radioactive waste could be reduced
- ✓ Furthermore, valuable resources such as copper which used in electric equipments can be recycled, and that contributes to the efficient use of resources
- Activation of copper materials used in electric equipments have not been evaluated

2. Objective

✓ Evaluate the radioactive concentration produced in copper material

3. Method

- (1) Analysis of target material composition
 - ✓ SEM/EDX and ICP-OES analysis for principal component
 - ✓ Activation analysis for minor component at UTR-KINKI
- (2) Estimation of neutron spectrum and flux at target position
 - ✓ Focused on thermal neutron
- (3) Evaluation of radioactivity concentration using PHITS3.17/DCHAIN-SP[1]

Ca

Sn

✓ Screening the radioisotopes to be evaluate

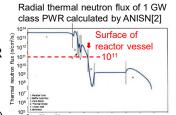
Composition of conductor Element Composition (wt%) Cu 99.95

 1.192×10^{-3}

0.051

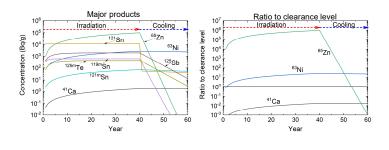
4. Results

- (1) Analysis of material composition
 - Principal component of conductor: Cu
 - ✓ Detected Ca and Sn as trace element
- (2) Estimation of neutron spectrum and flux at target position
 - Most of cables are located outside reactor vessel in PWR
 - → Focused on thermal neutron
 - Refer to a typical neutron flux
 - → 10¹¹ n/cm²/s EPRI report[2] (most conservative evaluation)



(3) Evaluation of radioactivity concentration

- √ 4¹Ca, 6³Ni and 6⁵Zn was generated
 (¹¹¹9, ¹²¹, ¹²¹mSn, ¹²²5Sb, ¹²²5mTe are excluded from clearance)
- ✓ Radioactive concentration of ⁶³Ni and ⁶⁵Zn are greater than the clearance criterion at 40 years



Target	T _{1/2}	CL (Bq/g)	40 y	50 y	60 y
⁴¹ Ca	1.03 × 10 ⁵ y	100	1.85	1.85	1.85
⁶³ Ni	101.2 y	100	2.71×10^{3}	2.53×10^{3}	2.36×10^{3}
⁶⁵ Zn	244.06 d	0.1	1.02×10^{5}	3.16	9.81×10^{-5}

5. Discussion

Composition

of cables

 Calculation results of PHITS3.17/DCHAIN-SP agree with analytical calculation

Criterion

Neutron

spectrum

Neutron flux

Activation

calculation

Radioactivity

concentration

Waste (mainly concrete)

Radiation controlled area

High level waste: Spent fuel
 Low level waste (L1, L2, L3): 6,000 t (≃2%)

(0.6%)

L2 (0.4%): Heat exchanger tube in Steam Generator L3 (0.6%): Bio shield (Concrete, Tube, ...)

Can be treated as non radioactive waste

About 500 kt (~98%) for 1.1 GW class PWR

2. Insulator

Cross sectional view of typical cables

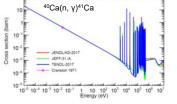
(0.3%)

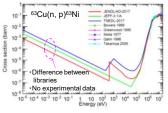
(0.1%)

L1 (0.1%): Control rod, Moderator,

- 63Ni is dominant in the residual radioactivity after 10 years from reactor shutdown
- ✓ Direct measurement of ⁶³Ni is difficult due to the low energy beta ray emission → Calculation based evaluation
- Uncertainty evaluation of activation cross section is important
- ✓ The cross section of ⁶³Cu(n, p)⁶³Ni have large difference between libraries → Important to uncertainty evaluation

	Concentration (Bq/g)		c_iN_A	(n, y) 65Zn 49.17% 787.5 mb 244.06 d	
Product	PHITS3.17/	Analytical	$A = \frac{c_i N_A}{M} \Phi \sigma (1 - e^{-\lambda t})$	β. 38.5%	
	DCHAIN-SP	solution	M: Atomic mass of target C _i : Isotopic composition N _a : Avogadro constant	63Cu (n, y) 64Cu (9.15% 4.057 b 12.7h	
⁴¹ Ca	1.85	1.90	λ: Decay constant of product		
⁶³ Ni	2.71×10^{3}	2.85×10^{3}	t: Irradiation time	(n, p) 17.96 µb	
⁶⁵ Zn	1.02×10^{5}	1.06 × 10 ⁵	_ (101.2 y	
102			1		





6. Conclusion

- Radioactive concentration of ⁶³Ni and ⁶⁵Zn in the cable near the reactor vessel may exceed the clearance criterion
- To improve the evaluation of radioactivity concentration, the uncertainty evaluation of activation cross section will be important

Reference

- [1] T. Sato, et. al., J. Nucl. Sci. Technol. 55, p.684-690 (2018).
- [2] A. Freitag, et. al., EPRI 1008018, (2003).
- [3] K.Shibata, et. al., JAEA-Conf 2016-004, p.47-52, (2016).